December 13, 1991

Docket No. 50-412
Serial No. BV-91-041

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Mr. J. D. Sieber, Vice President
Nuclear Group
Duquesne Light Company
Post Office Box 4
Shippingport, Pennsylvania 15077-0004

Dear Mr. Sieber:

SUBJECT: ISSUANCE OF AMENDMENT (TAC NO.M80042)

The Commission has issued the enclosed Amendment No. 41 to Facility Operating License No. NPF-73 for the Beaver Valley Power Station, Unit 2, in response to your application dated April 12, 1991.

The amendment modifies Technical Specification 3.3.3.8 by deleting a nonapplicable (first fuel cycle only) Action statement and reannotating the last two Action statements. It also modifies Table 3.3-11 by deleting a nonapplicable (first fuel cycle only) note.

An additional change to Table 3.3-11 that was proposed in your application would have reduced the total number of channels for the Reactor Vessel Level Indication System from 2 to 1. This change is not acceptable to the staff and therefore is denied.

A copy of the related Safety Evaluation is also enclosed. The Notice of Issuance will be included in the Commission's biweekly Federal Register notice. A separate Notice of Partial Denial of Amendment and Opportunity for Hearing is being forwarded to the Office of the Federal Register for publication.

Sincerely,

/s/
Albert W. De Agazio, Sr. Project Manager
Project Directorate I-4
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

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Enclosures:
1. Amendment No. 41 to NPF-73
2. Safety Evaluation
3. Notice of Partial Denial of Amendment and Opportunity for Hearing

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cc w/enclosures:
See next page

OFC	:PDI-4:LA	:PDI-4:PE	:PDI-4:PM	:PDI-4:D	:OGC
NAME	:SNorris	:JAndersen:cn	:ADeAgazio	:JStolz	:RBachmann
DATE	:11/18/91	:11/18/91	:11/18/91	:11/18/91	:11/21/91

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Document Name: 80042 AMENDMENT

DFO1

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

DUQUESNE LIGHT COMPANY

OHIO EDISON COMPANY

THE CLEVELAND ELECTRIC ILLUMINATING COMPANY

THE TOLEDO EDISON COMPANY

DOCKET NO. 50-412

BEAVER VALLEY POWER STATION, UNIT 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 41
License No. NPF-73

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment by Duquesne Light Company, et al. (the licensee) dated April 12, 1991, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

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2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. NPF-73 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 41 , and the Environmental Protection Plan contained in Appendix B, both of which are attached hereto are hereby incorporated in the license. DLCO shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of the date of its issuance, to be implemented within 60 days of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

John F. Stolz, Director
Project Directorate I-4
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: December 13, 1991

ATTACHMENT TO LICENSE AMENDMENT NO. 41

FACILITY OPERATING LICENSE NO. NPF-73

DOCKET NO. 50-412

Replace the following pages of Appendix A, Technical Specifications, with the enclosed pages as indicated. The revised pages are identified by amendment number and contain vertical lines indicating the areas of change.

Remove	Insert
3/4 3-57	3/4 3-57
3/4 3-58	3/4 3-58

INSTRUMENTATION

ACCIDENT MONITORING INSTRUMENTATION

LIMITING CONDITION FOR OPERATION

3.3.3.8 The accident monitoring instrumentation channels shown in Table 3.3.11 shall be OPERABLE.

APPLICABILITY: MODES 1, 2 and 3.

ACTION

a. With the number of OPERABLE accident monitoring instrumentation channels less than the Total Number of Channels shown in Table 3.3.11, either restore the inoperable channel(s) to OPERABLE status within 7 days or be in at least HOT SHUTDOWN within the next 12 hours except for the PORV(s) which may be isolated in accordance with Specification 3.4.11.

b. With the number of OPERABLE accident monitoring instrumentation channels less than the Minimum Channels OPERABLE requirements of Table 3.3.11, either restore the inoperable channel(s) to OPERABLE status within 48 hours or be in at least HOT SHUTDOWN within the next 12 hours.

c. With the number of OPERABLE Reactor Coolant System Subcooling Margin Monitor instrumentation channels less than the Minimum Channels OPERABLE requirements of Table 3.3.11, either restore the inoperable channel(s) to OPERABLE status within 7 days or be in a least HOT SHUTDOWN within the next 12 hours.

d. The provisions of Specification 3.0.4 are not applicable.

SURVEILLANCE REQUIREMENTS

4.3.3.8 Each accident monitoring instrumentation channel shall be demonstrated OPERABLE by performance of the CHANNEL CHECK and CHANNEL CALIBRATION operations at the frequencies shown in Table 4.3-7.

TABLE 3.3-11

ACCIDENT MONITORING INSTRUMENTATION

	INSTRUMENT	TOTAL NO. OF CHANNELS	MINIMUM CHANNELS OPERABLE	ACTION
1.	Pressurizer Water Level	3	2	a, b
2.	Auxiliary Feedwater Flow Rate	2 per steam generator	1 per steam generator	a, b
3.	Reactor Coolant System Subcooling Margin Monitor	2	1	c
4.	PORV Limit Switch Position Indicator	1/valve	0/valve	a, b
5.	PORV Block Valve Limit Switch Position Indicator	1/valve	0/valve	a, b
6.	Safety Valve Position Indicator	1/valve	0/valve	a, b
7.	Safety Valve Temperature Detector	1/valve	0/valve	a, b
8.	Containment Sump Wide Range Water Level	2	1	a, b
9.	Containment Wide-Range Pressure	2	1	a, b
10.	Reactor Vessel Level Indication System	2	1	a, b
11.	Core Exit Thermocouples	4/core quadrant	2/core quadrant	a, b

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NO. 41 TO FACILITY OPERATING LICENSE NO. NPF-73

DUQUESNE LIGHT COMPANY

OHIO EDISON COMPANY

THE CLEVELAND ELECTRIC ILLUMINATING COMPANY

THE TOLEDO EDISON COMPANY

BEAVER VALLEY POWER STATION, UNIT 2

DOCKET NO. 50-412

1.0 INTRODUCTION

By letter dated April 12, 1991, the Duquesne Light Company (the licensee) proposed certain changes to the Beaver Valley Power Station, Unit 2 Technical Specifications (TS). The proposed changes would (1) remove an obsolete action applicable to the first fuel cycle only from Action Statement TS 3.3.3.8, (2) remove an obsolete note relating to the first fuel cycle only from Table 3.3-11, and (3) reduce the total number of channels specified in Table 3.3-11 for the reactor vessel level indicating system (RVLIS).

2.0 BACKGROUND

NUREG-0737, "Clarification of TMI Action Plan Requirements," dated November 1, 1983, identifies those items for which Technical Specifications were required and included samples in Standard Technical Specification format with blanks or parenthesis appearing where the information was plant specific. RVLIS was included in this NUREG and was incorporated into the Beaver Valley Unit 2 Technical Specifications.

Before issuing NUREG-0737, the NRC staff solicited comments on the proposed Technical Specifications from all pressurized water reactor owners groups and the Atomic Industrial Forum, and appropriate comments were incorporated. Item 19 in the NUREG, Reactor Coolant Inventory Tracking System, required 2 channels, with 1 channel to be the minimum number operable.

Beaver Valley incorporated the format from the NUREG and required 2 channels of RVLIS, and that 1 be the minimum number operable. Due to the unproven RVLIS reliability, Beaver Valley TS 3.3.3.8 Action Statement c was incorporated to allow the plant to evaluate and correct any operating deficiencies related to RVLIS during cycle 1. The note on Table 3.3-11 also was provided to ensure it would be clearly understood that Action Statement c was only applicable to cycle 1.

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3.0 EVALUATION

The proposed change modifies TS 3.3.3.8 to eliminate an Action statement and note that is no longer applicable, and renames the remaining Action statements. Action Statement c and the note on Table 3.3-11 have been deleted since their applicability expired following cycle 1. Removing Action Statement c forces the renaming of actions d and e to maintain the consistency of TS requirements. The NRC staff has concluded that these changes are administrative in nature and are therefore acceptable.

The proposed change to reduce the total number of channels specified in Table 3.3-11 for RVLIS from 2 to 1 deviates from the 2 specified in Generic Letter 83-37. The NRC staff believes that RVLIS is a very useful system and that Technical Specifications and procedures are required for RVLIS. The RVLIS enhances the ability of plant operators to diagnose the approach of Inadequate Core Cooling (ICC) and to access the adequacy of actions taken to restore core cooling. The benefit is preventive in nature in that the instrumentation assists the operator in avoidance of a degraded or melted core when voids in the reactor coolant system and saturation conditions result from overcooling, steam generator tube rupture, and small break loss of coolant events. The addition of a reactor coolant inventory system, coupled with upgraded in-core thermocouple instruments and a subcooling margin monitor, provides an ICC instrumentation package which could reduce significantly the likelihood of incorrect operator diagnosis and errors for events such as steam generator tube ruptures, loss of instrument bus or control system upsets, pump seal failures, or overcooling events originating from disturbances in the secondary coolant side of the plant. For low probability events, involving coincidental multiple faults or more rapidly developing small break loss-of-coolant conditions, the ICC instrumentation could also reduce the probability of incorrect operator diagnosis and subsequent errors leading to a degraded core.

The reduction in RVLIS channels also is not consistent with the Westinghouse Standard Technical Specifications and the current requirement is not unique to Beaver Valley Unit 2. The reduction of channels from 2 to 1 for the RVLIS in the Beaver Valley Unit 1 Technical Specifications was an anomaly and does not justify the reduction in Unit 2. Therefore, based on the above discussion, the NRC staff has concluded that the request to reduce the total number of RVLIS channels from 2 to 1 cannot be approved as submitted.

4.0 STATE CONSULTATION

In accordance with the Commission's regulations, the Pennsylvania State official was notified of the proposed issuance of the amendment. The State official had no comments.

5.0 ENVIRONMENTAL CONSIDERATION

The amendment changes a requirement with respect to installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20. The NRC staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types,

of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendment involves no significant hazards consideration, and there has been no public comment on such finding (56 FR 41580). Accordingly, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendment.

6.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: James W. Andersen

Date: December 13, 1991

UNITED STATES NUCLEAR REGULATORY COMMISSION

DUQUESNE LIGHT COMPANY

OHIO EDISON COMPANY

THE CLEVELAND ELECTRIC ILLUMINATING COMPANY

THE TOLEDO EDISON COMPANY

DOCKET NO. 50-412

BEAVER VALLEY POWER STATION, UNIT 2

NOTICE OF PARTIAL DENIAL OF AMENDMENT TO FACILITY OPERATING LICENSE

AND OPPORTUNITY FOR HEARING

The U.S. Nuclear Regulatory Commission (the Commission) has denied, in part, a request by Duquesne Light Company, (DLC) for an amendment to Facility Operating License No. NPF-73, issued to DLC for operation of the Beaver Valley Power Station, Unit 2, located in Beaver County, Pennsylvania. Notice of Consideration of Issuance of this amendment was published in the FEDERAL REGISTER on August 20, 1991 (56 FR 41580).

The purpose of the DLC's amendment request was to revise the Technical Specifications (TS) by deleting a nonapplicable (first fuel cycle only) Action statement and reannotating the last two Action statements. It will also modify Table 3.3-11 by deleting a nonapplicable (first fuel cycle only) note.

Included in this proposal was a request to reduce the total number of channels specified in Table 3.3-11 for the Reactor Vessel Level Indication System (RVLIS) from 2 to 1. The proposed reduction in RVLIS channels is not consistent with Generic Letter 83-37 which specifies 2 channels, nor is it consistent with the Westinghouse Standard Technical Specifications. Furthermore, the current requirements for RVLIS are not unique to Beaver Valley

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Unit 2. Therefore, the NRC staff has concluded that your request to reduce the total number of RVLIS channels from 2 to 1 is not acceptable.

All other provisions of the amendment request have been approved by Amendment No. 41 dated December 13, 1991. Notice of Issuance of Amendment No. 41 will be published in the Commission's biweekly FEDERAL REGISTER notice.

The DLC was notified of the Commission's partial denial of the proposed change by letter dated December 13, 1991.

By January 27, 1992 , the DLC may demand a hearing with respect to the denial described above. Any person whose interest may be affected by this proceeding may file a written petition for leave to intervene.

A request for hearing or petition for leave to intervene must be filed with the Secretary of the Commission, U.S. Nuclear Regulatory Commission, Washington, DC 20555, Attention: Docketing and Service Branch, or may be delivered to the Commission's Public Document Room, the Gelman Building, 2120 L Street, NW, Washington, DC, by the above date.

A copy of any petitions should also be sent to the Office of the General Counsel, U.S. Nuclear Regulatory Commission, Washington, DC 20555, and to Gerald Charnoff, Esquire and Jay E. Silberg, Esquire, Shaw, Pittman, Potts & Trowbridge, 2300 N Street, NW, Washington, DC 20037, attorney for the DLC.

For further details with respect to this action, see (1) the application for amendment dated April 12, 1991, and (2) the Commission's letter to the DLC dated December 13, 1991.

These documents are available for public inspection at the Commission's Public Document Room, the Gelman Building, 2120 L Street, NW, Washington DC and at the local public document room located at the B. F. Jones Memorial Library,

663 Franklin Avenue, Aliquippa, Pennsylvania 15001. A copy of item (2) may be obtained upon request addressed to the U.S. Nuclear Regulatory Commission, Washington, DC 20555, Attention: Document Control Desk.

Dated at Rockville, Maryland, this 13th day of December 1991.

FOR THE NUCLEAR REGULATORY COMMISSION

John F. Stolz, Director
Project Directorate I-4
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation